November 9, 1984 NRC/TMI-84-079

MEMORANDUM FOR:

Harold R. Denton, Director

Office of Nuclear Reactor Regulation

Bernard J. Snyder, Program Director

THI Program Office

FROM:

William D. Travers, Deputy Program Director

TMI Program Office

SUBJECT:

NRC THI PROGRAM OFFICE NEEKLY STATUS REPORT FOR

NOVEMBER 4, 1984 - NOVEMBER 9, 1984

Data from effluent and environmental monitoring systems indicated no plant release in excess of regulatory limits. Waste processing continued on a routine basis. Plant parameters have shown no significant changes. Other site activities this period included: plenum assembly inspection, and continued fuel pool "A" refurbishment.

Significant items covered in the enclosure are:

Reactor Building Activities

Auxiliary and Fuel Handling Building Activities

Public Meetings

Summary sheets included in this report are:

- Liquid Effluent and Environmental Data
- Plant Status Data

ORIGINAL SIGNED BY:

William D. Travers Deputy Program Director TMI Program Office

Enclosure: As stated

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#### **ENCLOSURE**

#### REACTOR BUILDING ACTIVITIES:

The initial phase of the plenum inspection has been completed and there does not appear to be any condition which would preclude plenum jacking. All eight APSRs (axial power shaping rods) have been lowered into the rubble bed. The four 50-ton capacity jacks, which will be used simultaneously to initiate the lift of the 55-ton plenum, are being tested on a reactor vessel/plenum mockup. The plenum jacking is scheduled to occur in December 1984.

While workers are testing the plenum jacks and rehearsing the jacking procedures in the turbine building a remotely controlled robotic vehicle will be inspecting the basement of the reactor building. The remote vehicle will measure dose rates and provide closed circuit television video during its trek on the 282 ft. elevation of the reactor building. High radiation levels have prevented human access to the reactor building basement since the 1979 accident.

### AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES:

Refurbishment activities in the "A" fuel pool continued this week. Work presently being conducted is the decontamination of the stainless steel inner liner of the refueling pool.

The makeup and purification resin elution process has been on hold this week. The hold was caused by a valve failure on the "B" demineralizer eductor during startup adjustments. Maintenance work is currently in progress. Restart of the processing is expected about November 13, 1984.

# PUBLIC MEETINGS:

- On November 7, 1984, the NRC Commissioners met on the status of cleanup activities and funding for Three Mile Island Unit 2. Separate presentations were made by both the NRC staff and representatives of GPUN. Edwin E. Kintner, Executive Vice President for GPU Nuclear, provided a summary of the progress made thus far in the cleanup and indicated that removal of damaged fuel from the reactor is scheduled to begin in July 1985. The NRC staff noted its optimism regarding committed funding for the cleanup. While pointing out that there are many uncertainties associated with securing large funds from diverse sources, the staff stated that there is reasonable assurance that the licensee will be successful in securing the needed cleanup funding.
- 2. The Advisory Panel for the Decontamination of Three Mile Island Unit 2 met on November 8, 1984 in Lancaster, Pennsylvania. At this meeting the Panel received reports from the licensee and U.S. Environmental Protection Agency on the results of onsite and offsite Krypton-85 monitoring during the reactor pressure vessel head lift operation. The Department of Health, Commonwealth of Pennsylvania, presented a summary of health related studies conducted in the vicinity of the TMI site since the accident. The licensee also briefed the Panel on their investigation regarding misadjustment of the brakes on the reactor building polar crane.

The Panel's newest member, Mr. Joseph J. DiNunno attended this meeting. Mr. DiNunno, an independent consultant, holds B.S. and M.S. degrees in Electrical Engineering from Pennsylvania State University and the University of Maryland, respectively. He is a resident of Annapolis, Maryland and replaces Dr. Henry Wagner, also a Maryland resident, who has resigned from the Panel.

## APPENDIX 1

## LIQUID EFFLUENT AND ENVIRONMENTAL DATA

#### GPU Nuclear

Based on sampling and monitoring, liquid effluents from the TMI site released to the Susquehanna River were determined to be within regulatory limits and in accordance with NRC requirements and the City of Lancaster Agreement.

During the period November 2, 1984 through November 8, 1984, liquid effluents contained no detectable radioactivity at the discharge point. Individual effluent sources originating within Unit 2 contained minute amounts of radioactivity. Calculations indicate that less than 2.4 E-6 (0.0000024) of a curie of Cs-137 and less than 6.3 E-6 (0.0000063) of a curie of gross beta activity were discharged.

# Environmental Protection Agency

Lancaster Water Samples: 7 samples

Period Covered: October 21 - October 27, 1984

Results: Gamma Scan Negative for reactor related radioactivity

TMI Water Samples: 7 samples

Period Covered: October 20 - October 27, 1984

Results: Gamma Scan Negative for reactor related radioactivity

# NRC Environmental Data

The NRC operated continuous outdoor air sampler at the TMI site did not detect any reactor related radioactivity. The air sampler parameters are listed below. The analysis results were less than the lower limit of detectability of the analytical instruments: 8.6 E-14 uCi/cc for I-131 and 8.6 E-14 uCi/cc for Cs-137.

<u>Sample</u>	Period	<u>Volume</u>
HP-444	October 31 - November 7, 198	4 407.9 m <sup>3</sup>

#### APPENDIX 2

### PLANT STATUS

Reactor Vessel Configuration: Reactor vessel open with modified internals

indexing fixture installed

Core Cooling Mode: Heat transfer from the reactor coolant system (RCS)

to reactor building ambient

Available Core Cooling/Makeup Sources:

Standby pressure control (SPC) system

Reactor coolant bleed tank (RCBT) water transfer system

Mini decay heat removal (MDHR) system

Major Parameters as of 5:00 AM, November 9, 1984 (approximate values):

Reactor Coolant System:

Loop Temperatures:

Cold Leg (1) 67°F 69°F 69°F 69°F

Reactor Core:

Average Incore Thermocouples:\* 94°F Maximum Incore Thermocouple:\* 103°F

Decay Heat: 15 kilowatts

Reactor Building: Temperature: 61°F

Pressure: -0.05 psig

Airborne Radionuclide Concentrations:

Tritium: 3.5 E-8 uCi/cc (sample 10/29/84) Particulates: 6.4 E-10 uCi/cc (sample 11/7/84)

predominately Cs-137

<sup>\*</sup>Uncertainties exist as to the exact location and accuracy of these readings.